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OPEN Feasibility of a novel photoproduction of ²²⁵Ac and ²²⁷Th with natural thorium target

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We propose an innovative way to produce both ²²⁵Ac and ²²⁷Th, two precious radioisotopes enabling promising targeted alpha therapy, in a natural thorium target bombarded with a 30-90 MeV electron beam. Bremsstrahlung photons in the target are analyzed by MCNP and in-situ photonuclear transmutation of ²³²Th is evaluated by using the TENDL nuclear data. In the photo-transmutation analysis, 13 nuclides including ²²⁹Th and ²³¹Pa are modelled. Special procedures with chemical separations are also proposed to produce pure ²²⁵Ac and ²²⁷Th in separate streams. In addition, performance of the new approach is compared with conventional methods in terms of the ²²⁵Ac and ²²⁷Th yields. After a Th target is bombarded with a 500 kW electron beam for a year, yearly ²²⁵Ac yield is ~ 8.47 GBg (semi-permanently) and yearly ²²⁷Th yield is ~ 48.9 GBg over 50 years, and their yields are at least doubled in a 2-year irradiation. This work will help increase global supply of the two precious isotopes and would invariably help advance TAT-related researches and developments.

Targeted alpha (α) therapy (TAT) is becoming a promising way to treat incurable cancers¹. An α -emitter conjugated with a suitable carrier can transfer lethal energy to cancer cells precisely even when the tumor tissue is tiny. TAT minimizes damages in normal tissues due to high linear energy transfer ($60-230 \text{ keV}/\mu m$) and short range (50–90 μ m) of α -particles in human body. Demand for α -emitters has been increasing since the US FDA approved ²²³Ra chloride as a radiopharmaceutical. For cancer treatments, both ²²⁵Ac and ²²⁷Th are regarded as best α -emitters due to an adequate half-life and emission of multiple α -particles by daughter nuclides. After an outstanding efficacy was reported for ²²⁵Ac in treating prostate cancers, a lot of researches on TAT with ²²⁵Ac are underway globally. Meanwhile, ²²⁷Th is also getting lots of attention due to its chemical affinity to be easily bonded with various antibodies. In particular, ²²⁷Th decays to ²²³Ra, which is an officially approved α -emitter for clinical use, and it is rather free from regulatory issues other than its daughter²⁻⁴. Radiopharmaceuticals labelled with ²²⁷Th are being tested for various cancer treatments as well^{2,5}.

Despite of the high potential of ²²⁵Ac and ²²⁷Th, lack of their supply is impeding progress in radiopharmaceutical for TAT. Even for ²²⁵Ac, the annual supply is only about one thirtieth of its demand that reached about 1850 GBq in 2019⁶. The only commercial way to produce ²²⁵Ac is a milking from ²²⁹Th cow which is a decay product of the legacy ²³³U stockpile^{6,7}. It is known that the stockpile was mainly produced in the Th-fuelled nuclear reactors in the 1960s in US and Russia^{8,9}. Consequently, the ²²⁵Ac supply is rather monopolized and affordable ways are strongly required for progress in the ²²⁵Ac-based TAT.

Researches are currently underway to find out alternative methods to produce ²²⁵Ac. For example, the Tri-lab project team is looking into ways to directly extract ²²⁵Ac from a proton-irradiated thorium target, and produced 81.4 GBq of 225 Ac in a 10-day irradiation of 200 MeV proton beam at 165 μ A^{10,11}. Regarding this method, one concern is ~0.3% yield of ²²⁷Ac contaminating ²²⁵Ac in addition to the relatively high proton energy required. It is known that ²²⁷Ac may cause severe level of radiation dose for the rest of patient's lives if it is injected with ²²⁵Ac¹². Meanwhile, utilizing ²²⁶Ra(p,2n)²²⁵Ac reaction is regarded as the most feasible approach due to a low proton energy. Institute for Transuranium Elements (ITU) produced ~ 485 MBq of ²²⁵Ac by irradiating a RaCl₂ target with a 28 MeV beam for 45.3 h at 50 µA beam current^{7,13}. However, it is known that procurement and treatment of the highly radioactive Ra target are challenging and costly. Though other methods including irradiation in nuclear reactors have been investigated to utilize 237 Th(n, γ) 233 U and 226 Ra(n,2n) 225 Ra reactions, low yield of 225 Ac and ²²⁷Ac impurity catches up with their commercialization⁷.

The situation is even worse in the case of ²²⁷Th production. The natural ²²⁷Ac yield from ²³⁵U is very small due to long half-life of ²³⁵U and ²²⁷Th briefly exists as a daughter nuclide of ²²⁷Ac. Currently, there are no other

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Figure 1. Schematic diagram of thorium target system.



Figure 2. Photonulcear reactions and decays in Th target during electron beam irradiation.

options but to produce ²²⁷Ac from the ²²⁶Ra(n,γ)²²⁷Ra reaction^{2,14}, which means that the aforementioned Ra target issue remains unresolved and a costly neutron source is required.

In response to the high demand of the 'rarest drugs' ²²⁵Ac and ²²⁷Th, we propose an innovative method to semi-permanently produce the α -emitters with an electron accelerator and natural thorium (²³²Th) target. The Monte Carlo N-Particle transport (MCNP) version 6.2 code is used to calculate X-ray or photon generation and heat deposition in the Th target¹⁵. The photo-transmutations of ²³²Th into ²²⁵Ac or ²²⁷Th are estimated by using the TALYS-generated Evaluated Nuclear Data Libraries (TENDL) cross-sections for various (γ ,xn) photonuclear reactions¹⁶.

Methods

Photonuclear reactions in thorium target. Figure 1 shows schematic diagram of the proposed Th target system for photo-production of both ²²⁵Ac and ²²⁷Th. A metallic cylindrical Th target is directly bombarded by an electron beam to generate Bremsstrahlung photons and trigger photonuclear reactions. The target should rotate to manage locally-deposited heat during beam irradiation and it is supposed to be equipped with a cooling and radiation protection system as in the TRIUMF Isotope Separator and Accelerator (ISAC) facility¹⁷. Taking into account target cooling, the electron beam power is set to 500 kW and a relatively large target is considered (radius = 10 cm and thickness = 6 cm) to minimize the photon leakage from the target. To achieve appropriate target cooling, we adopt a beam diameter of 40 mm and a rotational speed of 120 RPM (rotation per minute). Detailed discussion on the heat deposition and removal is provided in Supplementary Information Sect. A.

Figure 2 describes major photonuclear reactions and transmutation chains of 13 nuclides in the Th target. Two most significant pathways to generate ²²⁵Ac and ²²⁷Th are highlighted with bold lines respectively: ²³²Th(γ , 3n)²²⁹Th \rightarrow ²²⁵Ra \rightarrow ²²⁵Ac and ²³²Th(γ , n)²³¹Th \rightarrow ²³¹Pa \rightarrow ²²⁷Ac \rightarrow ²²⁷Th. Isotopic yields involving multiple photonuclear reactions with other thorium isotopes are actually very small. Other actinium isotopes can also be



Figure 3. Comparison between TENDL and experimental data for (γ,xn) cross-sections of ²³²Th.

generated from (γ ,p) and (γ ,p) photonuclear reactions. However, they quickly decay to thorium isotopes except for ²²⁹Th(γ ,np)²²⁷Ac reaction, which is insignificant due to very small (γ ,np) cross-section. We also neglect the photo-fission of ²³²Th, in spite of noticeable probability, since both fission products and neutrons hardly affect yield of ²²⁵Ac and ²²⁷Th. Meanwhile, the impact of photo-fissions on heat deposition turns out to be noticeable and it is discussed in Supplementary Information Sect. A. In the Th target, many fast neutrons are also generated by the photonuclear reactions including the photo-fissions. When the Th target is bombarded with 500 kW electron beam (70 MeV energy and 7.14 mA current), the neutron production is roughly estimated to be ~ 1.94E + 15 #/s. However, it is found that most of the fast neutrons (>97%) simply escape the target and cannot contribute to the production of ²²⁵Ac and ²²⁷Th.

Measured cross-sections of the photonuclear reactions of the Th and Pa isotopes are unknown except for (γ,n) and $(\gamma,2n)$ reactions of ²³²Th¹⁸. In order to calculate the photonuclear transmutation rate, the TENDL nuclear data are utilized. Figure 3 shows experimental (γ,xn) and photo-fission cross-sections of ²³²Th together with the TENDL data for comparison. Except for overrated TENDL data of (γ,n) cross-section at a high energy tail, most of the experimental data for (γ,n) and $(\gamma,2n)$ reactions are within one standard deviation of the TENDL data. While nuclear reaction modelling codes including TALYS generally provide rather uncertain parameters for photonuclear reactions, it is known that the parameter of giant dipole resonance (GDR) reactions such as (γ,xn) of heavy nuclides is slowly varying with the atomic mass number and the deviation from one nuclide to another is rather small^{19–21}. Therefore, a GDR cross-section of a nuclide can be well predicted based on the experimental data of neighboring nuclei. Based on the measured cross-sections and general credibility of the TENDL library as shown in Fig. 3, actual cross-sections of 2^{232} Th $(\gamma,3n)^{229}$ Th and other (γ,xn) reactions with similar mass number such as 2^{31} Pa are expected to be similar to the TENDL data. Impacts of TENDL data uncertainty on the isotopic yields are given in Supplementary Information Sect. E.

As indicated in Fig. 3, high energy (>6 MeV) photons are required for the photonuclear (γ ,xn) reactions of ²³²Th and the electron energy should exceed about 30 MeV. The Bremsstrahlung photon source in the Th target is generated by using the well-validated MCNP code. For a given photon source, number density of 'A' nuclide (N^4) is estimated by solving the following balance equations.

$$\frac{dN^A}{dt} = -\lambda^A_{Nat}N^A - \lambda^A_{\gamma}N^A + \lambda^{Other}_{Nat}N^{Other} + \lambda^{Other}_{(\gamma,xn)}N^{Other},$$
(1)

$$\lambda^{A}_{(\gamma,\mathbf{x}n)} = \left(\int \sigma^{A}_{(\gamma,\mathbf{x}n)} \frac{dN_{\gamma}}{dE} dE\right), \ \lambda^{A}_{\gamma} = \sum_{\mathbf{x}=1}^{3} \lambda^{A}_{(\gamma,\mathbf{x}n)}, \tag{2}$$

where λ_{Nat} is natural decay constant, $\lambda_{(\gamma,xn)}$ is an effective decay constant for a (γ,xn) reaction, λ_{γ} is summation of the effective decay constants for all (γ,xn) reactions, and dN_{γ}/dE is the photon spectral density obtained by MCNP 6.2. A space-dependent photon spectral density should be used for accurate evaluation of the isotopic yields. The last two terms in Eq. (1) denote production of 'A' nuclide due to decay or transmutation of other nuclides. Detailed coupled equations for the 13 nuclides in Fig. 2 are described in Supplementary Information



Figure 4. Procedure for semi-permanent milking of ²²⁵Ac and ²²⁷Th.

Sect. B. Spatial distribution of the photon source in Eq. (2) is pre-evaluated with MCNP and it is assumed to be constant since transmutation rate of 232 Th is extremely low in the rotating Th target.

Procedures to retrieve pure²²⁷**Th and**²²⁵**Ac.** To produce pure ²²⁵Ac and ²²⁷Th from the irradiated target, their parent nuclides, ²²⁹Th and ²²⁷Ac, should be separated in order to avoid the ²²⁷Ac contamination. There have been a few experimental validations for extraction of ²²⁵Ac from ²³³U or ²²⁹Th stockpiles^{22,23}. For the same purpose, we propose a simple strategy and chemical processings for periodic milking of the α -emitters, as shown in Fig. 4.

Purpose of the cooling phase is to let ²³¹Th (progenitor of ²²⁷Ac) completely decay out. After irradiation, the target should be cooled for a certain time to minimize the ²²⁷Ac impurity for pure ²²⁵Ac extraction. Once ²³¹Th disappears by its natural decay, all remaining Th isotopes are chemically separated from the others, then ²²⁵Ac is purely produced by decay of ²²⁹Th and this chemical extraction of ²²⁵Ac can be near permanent due to long half-life of ²²⁹Th. It should be noted that ²²⁶Ra can be also produced due to decay of ²³⁰Th in the separated Th stream and it can be used as the target material for other accelerated-based ²²⁵Ac production methods.

Meanwhile, pure ²²⁷Th can be chemically extracted from the separated irradiation byproducts, which are basically Ra, Ac, and Pa isotopes, i.e., ²³¹Pa slowly decays to ²²⁷Ac decaying again to ²²⁷Th with a half-life of ~ 22 years. If ²²⁷Th is directly extracted from the separated byproducts without any additional chemical partitionings, the ²²⁷Th can be blended with ²³⁰Th generated by a natural decay of ²³⁰Pa, which may cause an unwanted hazard induced by its daughter nuclides. For the purity of ²²⁷Th, we chemically partition the byproducts into three parts: Ra, Ac, and Pa isotopes (blue lines in the non-Th products in Fig. 4). Then, ²²⁷Ac is extracted from the separated Pa and moved to the actinium part where pure ²²⁷Th can be purely extracted periodically. It is important to note that ²²⁷Th production is quite semi-permanent (over 50 years) since half-life of ²²⁷Ac is about 22 years. All chemical procedures in Fig. 4 are very analogous with ones used in the current ²²⁵Ac extraction from natural thorium target bombarded by proton beam^{24,25}, and they are considered to be readily available. It is also obvious that the ²²⁵Ac and ²²⁷Th yields should increase if the amount of ²²⁹Th and ²³¹ Pa in the target is increased with a prolonged beam irradiation and/or higher electron current.

Results

Parameter optimization for electron beam. A 500 kW electron beam is considered to evaluate the feasibility and performance of the new photo-production of ²²⁵Ac and ²²⁷Th. In order to find optimal parameters for the 500 kW electron beam, isotopic yield of ²²⁹Th and ²³¹Pa (parent nuclei of ²²⁵Ac and ²²⁷Th) is evaluated for different beam diameters and electron energies. For accurate evaluation of the photon spatial distribution, the target is subdivided into small zones in both radial and axial directions, which is described in detail in Supplementary Information Sect. C.

For the sensitivity analysis, it is assumed that a 70 MeV beam bombards a fixed target with a 5 cm radius and a 6 cm height. The target size is large enough to minimize the leakage of Bremsstrahlung photons from the target. Figure 5 shows the spatial distribution of isotopic yields with different electron beam sizes (yellow area) after a one-year beam irradiation and cooling of a month. The spatial isotopic yield is evaluated with a different spatial discretization of the target depending on the beam size, as indicated by the dotted lines in Fig. 5. One

a	Beam diameter: 15 mm				Pa-231 Th-229	b Beam diameter: 40 mm					
0	117.88 8.85	121.06 7.27	60.79 2.78	27.69 0.96	12.56 0.33	0	58.50 3.98	131.02 8.72	88.91 5.01	40.29 1.72	17.72 0.58
د ج	261.44 23.50	226.74 15.61	94.91 4.69	38.22 1.36	16.05 0.43	Thickness (cm)	119.77 9.57	267.40 21.02	160.51 10.47	61,42 2.88	24.13 0.81
u) ss	583.12 64,79	398.49 31.28	131.35 6.77	46.47 1.64	17.97 0.47		242.16 22,95	544.64 51,39	276.20 20,92	83.95 4.17	28.74 0.98
Thickne	1299.44 190.11	615.31 53,84	147.63 7.28	46.25 1.53	17.15 0.42		478.41 55.20	1108.15 128.35	438.08 38,33	94.68 4.66	28.78 0.93
	2816.44 613.09	680.06 60.12	115.29 4.88	35.48 1.05	13.51 0.30		907.61 132.39	2252.19 331.70	569.80 56142	75,26 3.19	22.36 0.63
1	3104.67 922.41	233.99 14.95	53.72 1.89	20.52 0.56	8.84 0.20		902.06 170.35	2480.11 476.56	267.55 27,74	35.42 1.22	13.17 0.34
0 1 2 3 4 5 Radius (cm)						5 (0 1 2 3 4 Radius (cm)				

Figure 5. Isotopic yield (MBq) in a fixed target with different beam diameters.

Beam diameter: 40 mm											
6	1.29	5.75	18.26	48.03	76.92	84.88	64.31	30.25	12.34	9.78	
5	0.04	0.21	0.90	2.85	4.84	5.29	3.83	1.47	0.46	0.27	
5	1.71	8.21	30.36	91.46	151.22	166.72	122.54	50.25	17.68	12.67	
4	0.05	0.33	1.73	6.47	11.21	12.27	8.66	2.81	0.70	0.36	
(m ⁴)	2.00	10.49	47.47	172.64	295.41	324.97	231.51	78.44	22.66	14.56	
sss (e	0.05	0.44	3.11	14.54	25.90	28.41	19.51	5.05	0.94	0.43	
ckne	1.96	11.29	66.86	323.68	569.06	627.15	433.27	109.82	24.15	14.29	
Thi	0.04	0.35	4.94	33.06	59.72	65.49	43.99	8.03	0.99	0.39	
2	1.53	8.92	73.93	602.36	1073.37	1182.89	802.16	119.77	18.98	11.16	
	0.03	0.30	5.89	75.55	137.05	150.29	100.45	9.52	0.68	0.27	
1	0.92	4.58	31.16	606.80	1069.30	1180.30	807.34	50.28	9.92	7.06	
~	0.01	0.06	2.18	101.56	180.75	198.58	135.16	3.65	0.31	0.17	
0) 1	1 2	2 3	3 4	4 5	5 6	5 7	7 8	3 9	9 10	
Zone A Zone C Radius (cm)											
Zone B I Zone D											



notes that total yield of ²²⁹Th drops by ~ 22% when beam diameter is increased from 15 to 40 mm, whereas the 40 mm beam enhances ²³¹Pa production slightly. The clearly lower ²²⁹Th yield with the 40 mm beam is because contribution of the ²³¹Pa(γ ,2n)²²⁹Pa reaction can be noticeably lowered due to lower concentration of ²³¹Pa with a larger beam diameter. Meanwhile, the slightly higher ²³¹Pa production for the 40 mm beam is simply due to the slower depletion rate of ²³²Th in a bigger irradiated region. It is clear that a smaller beam size is favorable in terms of the ²²⁹Th yield if the integrity of the target is guaranteed during irradiation.

In the actual rotating Th target as shown in Fig. 1, the effective beam size or irradiated area is significantly increased compared with a fixed target. In the Th target with a 120 RPM speed, it is assumed that the spatial photon spectral density is uniform in the azimuthal direction and they are evaluated with a detailed discretization in the radial direction. The isotopic yields of ²²⁹Th and ²³¹ Pa are given in Fig. 6 when the target radius is 10 cm

Beam diameter (target condition)	15 mm (fix)	40 mm (fix)	40 mm (rotating)
²²⁹ Th	2.04	1.59	1.48
²³¹ Pa	11.36	11.82	12.05



 Table 1. Isotopic yield (GBq) with different beam irradiation conditions.

Figure 7. Zone-wise photon spectral densities for 70 MeV electron beam and isotopic yield of ²²⁹Th and ²³¹ Pa as a function of electron energy.

0,

and beam diameter is 40 mm. It is noted that total yield of ²²⁹Th is slightly lower than that of the fixed target due to further dispersion of the electron beam, and production of ²³¹ Pa is slightly increased. The isotopic yields in the rotating target are compared with ones in the fixed target in Table 1.

Figure 7a shows MCNP-evaluated zone-wise photon spectral densities for the rotational target in Fig. 6. One clearly notes that the photon flux is highest in the zone A and decreases more quickly in the radial direction due to a localized electron distribution. The highly position-dependent photon density results in the space-dependent yield distribution in Fig. 6. The number of electrons in the MCNP simulation is 8,000,000, leading to accurate photon spectral density in the whole target region. It is mentioned that impact of uncertainty of the photon flux on the α -emitter yield is less than 0.6%.

Figure 7b shows the isotopic yield of ²²⁹Th (source of ²²⁵Ac) and ²³¹Pa (source of ²²⁷Th) in the whole target as a function of the electron energy after a one-year beam irradiation and cooling of a month. The yield for both isotopes converges to an asymptotic value with electron energy since the photonuclear cross-sections of ²³²Th are maximized below 30 MeV as indicated in Fig. 3. The optimal energy is about 70 MeV for the ²³¹Pa, while it is slightly above 90 MeV for ²²⁹Th. This is because ²²⁹Th production is mainly dominated by the (γ ,3n) reaction requiring higher electron energy than in the (γ ,n)-dominating ²³¹Pa production. Based on the results, 70 MeV is considered to be near optimal to produce the two isotopes simultaneously as incremental yield of ²²⁹Th is marginal above 70 MeV.

Periodic yield of ²²⁵**Ac and** ²²⁷**Th.** Figure 8 shows time-dependent isotope-wise number density in the rotating target and activity of important nuclides in the periodic extractions. The ²²⁷Ac can be less than 10^{-8} % in the separated Th stream after 30-day cooling as shown in Fig. 8a. Nuclides inventories in the separated Th and non-Th products for periodic extractions of ²²⁵Ac and ²²⁷Th are shown in Fig. 8b,c. Long-term extraction of both α -emitters can be realized due to a sufficient amount of their parent nuclei and it is clear that extraction of pure ²²⁵Ac and ²²⁷Th is possible in each separated stream. Optimal extraction or milking period for ²²⁵Ac and ²²⁷Th is determined so that accumulated yield should be maximized, as shown in Fig. 8d.

When ²²⁵Ac is extracted for 50 years after a one-year beam irradiation, its accumulated yield increases until 31-day cooling period and gradually dwindles. Meanwhile, there is no local maximum point of the accumulated ²²⁷Th yield with different extraction periods due to a large difference of half-life between ²²⁷Th and its parent nuclide, and basically the shorter the extraction, the better in terms of the accumulated yield. In this work, we regard 1 day for ²²⁷Th and 31 days for ²²⁵Ac as optimum extraction period.

Figure 9 shows the yield of ²²⁵Ac and ²²⁷Th per an extraction for three irradiation periods with a 70 MeV beam over a 50-year operation. One observes in Fig. 8 that ²²⁵Ac yield is almost constant, while the ²²⁷Th yield increases over 30 years and then slowly decreases due to the longer half-life of ²²⁷Ac than that of ²²⁵Ra decaying to ²²⁵Ac, which is the root cause of the low ²²⁷Th yield right after irradiation. If a daily ²²⁷Th yield is too small, a longer extraction period can be adopted in the early operational period (See Supplementary Information Sect. D).



Figure 8. Isotopic evolution in Th target, periodic extraction and accumulated yield of ²²⁵Ac and ²²⁷Th.



Figure 9. Yield of ²²⁵Ac and ²²⁷Th per extraction with different beam irradiation times.

Discussion. Table 2 compares the proposed method with other approaches using a ²²⁶Ra target to produce ²²⁵Ac and ²²⁷Th. Performances of the new method are based on the mean TENDL data. Their variations due to uncertain TENDL data are discussed in Supplementary Information Sect. E . Regarding the existing methods, an optimistic ²²⁵Ac yield is also evaluated by improving beam and target parameters^{13,26,27}. For an optimistic ²²⁷Th yield of the reactor-based approach¹⁴, an optimal irradiation time and a relatively large target are considered (see Supplementary Information Sect. F).

The new method requires only one 'long' irradiation and ²²⁵Ac can be extracted semi-permanently. However, a 'short' irradiation is always necessary for each production of ²²⁵Ac in the others. In order to compare a yearly yield, it is assumed, based on engineering judgements, that there should be at least 12-h preparation time between

Facility		Nuclear reactor	Proton accelerator		Electron accelerator				
Governing reaction		²²⁶ Ra(n, γ) ²²⁷ Ra		²²⁶ Ra(p,2n) ²²⁵ Ac		²²⁶ Ra(y,n) ²²⁵ Ra		This work	
Case		Exp ¹	Potential ²	Exp ¹	Potential ²	Exp ¹	Potential ²	Potential ²	
Neutron flux or beam	current	$1.09 \times 10^{14} \text{ n/cm}^2\text{-s}$		50 µA	500 µA	26 μΑ		7.14 mA	
Irradiation time (day)		11.7	540	1.89	0.042	0.121		365	
Particle energy		<10 keV		28 MeV		18 MeV		70 MeV	
Target mass		5.1 μg	5 g	30 mg	210 mg	40 mg	2.52 g	22 kg	
Yield per irradiation	²²⁵ Ac	-	-	484.7	743.7	1.07	67.7	Single irradiation	
(MBq)	²²⁷ Th	Single irradiation		-	-	-	-	Single irradiation	
Yearly yield	²²⁵ Ac	-	-	74.1	501.1	0.63	39.8	8.47 (17.1) ⁴	
(GBq)	²²⁷ Th ³	0.0004	1440	-	-	-	-	48.9 (100.1) ⁴	

Table 2. Comparison of ²²⁵Ac and ²²⁷Th production methods. ¹Experiment. ²Optimistic expectation. ³50-year average yield. ⁴2-year irradiation.

irradiations in the conventional methods. The yearly ²²⁷Th yield is averaged over a 50-year operation since it is time-dependent in both new and reactor-based approaches.

The optimistic reactor-based ²²⁷Th yield is much higher than that of the new method due to the very favorable target and irradiation conditions. Unfortunately, the nuclear reactor is extremely costly and is not available in many countries. Although the yearly ²²⁵Ac yield of the new method is lower than the optimistic expectation of the others, the following advantages should be recalled: (1) pure ²²⁵Ac can be produced in the new photo-production, while there should be some level of ²²⁷Ac contamination in methods utilizing a Th target bombarded by a proton beam, (2) a semi-permanent source is produced after a single irradiation in a compact electron accelerator, (3) both ²²⁵Ac and ²²⁷Th are simultaneously produced. It is also noteworthy that both yields are rather proportional to the irradiation time since contribution of multiple photonuclear reactions to the yields is quite marginal in the rotational Th target.

Conclusions

The newly proposed ²²⁵Ac and ²²⁷Th photo-production method is deemed to be viable and promising in several aspects. First of all, it can produce pure ²²⁵Ac and ²²⁷Th by irradiating a 'cheap' natural Th target with a 50–70 MeV electron beam and then using the conventional chemical procedures. The electron energy needs to be over 50 MeV and 70 MeV turns out to be optimal for the application. To maximize the ²²⁵Ac and ²²⁷Th yield, a rotational Th target is required and it is shown that yearly yield of ²²⁵Ac and ²²⁷Th can be ~ 8.5 GBq and 49 GBq, respectively, for a 500 kW beam. The required electron accelerator is easily available and the new method can get all counties to benefit from TAT. More importantly ²²⁵Ac production is almost permanent and ²²⁷Th can be produced much over 50 years after a beam irradiation. Yearly ²²⁷Th yield is about four times higher than that of ²²⁵Ac, and the new method is particularly useful for production of emerging ²²⁷Th. We believe that this work will help increase global supply of the two precious isotopes and would invariably help advance TAT-related researches and developments.

Taking into account the photonuclear cross-sections, the expected ²²⁷Th yield seems to be rather reliable while the ²²⁵Ac yield is more uncertain due to absence of measurement data for the governing (γ ,3n) cross-section of ²³²Th. Therefore, the ²³²Th(γ ,3n)²²⁹Th cross-section should be experimentally evaluated for more concrete conclusions.

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Author contributions

K.J. and Y.K. wrote and reviewed the main manuscripts and Supplementary Material. All the Figures and Tables in them were prepared and drawn by K.J.

Competing interests

The authors declare no competing interests.

Additional information

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